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Title: **MCNP Progress & Performance Improvements**

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MCNP Progress & Performance Improvements

Forrest B. Brown, Jeffrey S. Bull, Michael E. Rising

Monte Carlo Codes, XCP-3
Los Alamos National Laboratory

US DOE/NNSA Nuclear Criticality Safety Program –

What have we done for you lately (FY 2014, FY 2015) ?

- **MCNP6.1.1 Release, with ENDF/B-VII.1**
- **Verification / Validation**
- **User Support & Training**
- **Performance Improvements**
- **Work in Progress**

MCNP6

MCNP6 Status (1)

- **MCNP6 releases by RSICC**

MCNP6.1 – 2013, production version

MCNP6.1.1 – 2014, same criticality, faster, beta features for DHS

Nuclear Data – ENDF/B-VII.1 data, updates, & older data

Reference Collection – 700+ technical reports

V&V Test Collection – 1434 test problems

12,000+ copies of MCNP5 distributed by RSICC

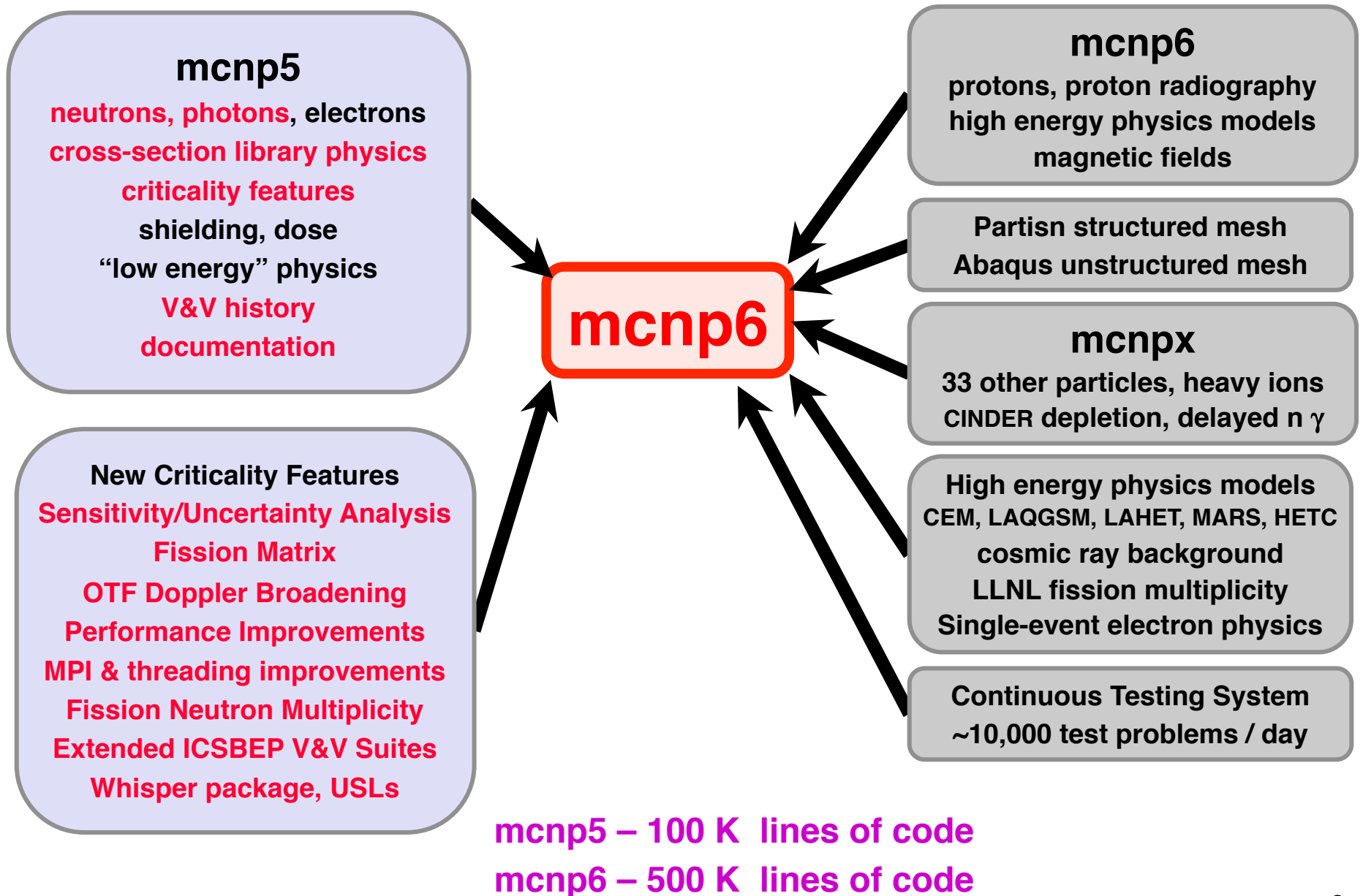
7,819+ copies of MCNP6 distributed by RSICC

- **MCNP5 is frozen & unsupported. Last version 2010.**
- **Criticality safety community needs to transition to MCNP6**



Support from DOE-NNSA-NCSP, DOE-NNSA-ASC,
DOE, DOD, DTRA, DHS/DNDO, NASA, & others

MCNP6 Status (2)



Verification & Validation

We do a lot of verification/validation work - all the time:

MCNP Verification-Validation, 100+ reports on MCNP Website

MCNP6 Optimization & Testing for Criticality Safety Calculations, LA-UR-15-20422 (2015).

Validation of MCNP6.1 for Criticality Safety of Pu-Metal, -Solution, and -Oxide Systems, LA-UR-14-23352 (2014).

Verification of MCNP6.1 & MCNP6.1.1 for Criticality Safety Applications, LA-UR-14-22480 (2014).

Verification of MCNP5-1.60 and MCNP6.1 for Criticality Safety Applications, LA-UR-13-22196 (2013).

Verification of MCNP5-1.60 and MCNP6-Beta-2 for Criticality Safety Applications, LA-UR-12-210 (2012).

MCNP5-1.60 Release & Verification, Trans Am Nuc Soc 104, June 2011, LA-UR-11-00230 (2011).

ENDF/B-VII.1 Neutron Cross Section Data Testing with Critical Assembly Benchmarks & Reactor Experiments, Nuclear Data Sheets, Vol 112, No. 12, 2997-3036 [LA-UR-11-11271] (2011).

An Expanded Criticality Validation Suite for MCNP, ICNC-2011, LA-UR-11-04170 (2011).

Verification of MCNP5-1.60, LA-UR-10-05611 (2010).

Nuclear Data

Listing of Available ACE Data Tables, LA-UR-13-21822, rev 4 (2014)

Continuous Energy Neutron Cross Section Data ...ENDF/B-VII.1, LA-UR-13-20137 (2013).

LANL Data Testing Support for ENDF/B-VII.1, LA-UR-12-20002 (2012).

ENDF/B-VII.1 Nuclear Data....., Nuclear Data Sheets, Vol 112, No. 12, 2887-2996 (2011).

ENDF/B-VII.0: ... Nuclear Data ..., Nuclear Data Sheets, Vol. 107, Number 12 (2006)

New ACE-Formatted Neutron and Proton Libraries Based on ENDF/B-VII.0, LA-UR-08-1999 (2008).

Release of New MCNP S(α,β) Library ... ENDF/B-VII.0, LA-UR-08-3628 (2008).

Verification & Validation (2)

Table 1. MCNP6.1 and MCNP6.1.1-Beta Results
for Analytic Keff Benchmarks

Case	Name	Analytic keff	MCNP_Results keff	std
prob11	Ua-1-0-IN	2.25000	2.25000	0.00000
prob14	Ua-1-0-SP	1.00000	1.00006	0.00010
prob18	Uc-H2O(2)-1-0-SP	1.00000	1.00005	0.00011
prob23	UD2O-1-0-CY	1.00000	1.00000	0.00006
prob32	PUa-1-1-SL	1.00000	0.99995	0.00011
prob41	UD2Ob-1-1-SP	1.00000	1.00003	0.00007
prob44	PU-2-0-IN	2.68377	2.68377	0.00003
prob54	URRa-2-0-SL	1.00000	1.00007	0.00013
prob63	URRd-H2Ob(1)-2-0-ISLC	1.00000	0.99993	0.00006
prob75	URR-6-0-IN	1.60000	1.59999	0.00001

Results are identical for MCNP6.1 and MCNP6.1.1-Beta.

Wall-clock time, using 8 threads on Mac Pro:

MCNP6.1 151 min

MCNP6.1.1-beta 87 min

From LA-UR-14-22480

Verification & Validation (3)

From LA-UR-14-22480 (2014), using Intel-12 compiler for all codes:

VERIFICATION_KEFF Suite

- MCNP6.1 & MCNP6.1.1:

– analytic problems with exact K_{eff} results

All results match

VALIDATION_CRITICALITY Suite

- MCNP5, MCNP6.1, MCNP6.1.1:

– 31 ICSBEP Cases, ENDF/B-VII.0

All results match

VALIDATION_CRIT_EXPANDED Suite

Shortened Problems

- MCNP5, MCNP6.1, MCNP6.1.1:

– 119 ICSBEP Cases, ENDF/B-VII.0

All results match

Standard Problems

- MCNP5, MCNP6.1, MCNP6.1.1:

4 diffs, within statistics

VALIDATION_CRIT_WHISPER Suite

- Used for LANL NCS validation of MCNP6.1 & ENDF/B-VII.1, and determining baseline USLs for Pu-metal, -solution, & -oxide systems
- Includes sensitivity profiles for all reactions/isotopes/problems
- Will be added to standard MCNP Criticality V&V suites in 2015

– 1086 ICSBEP Cases, ENDF/B-VII.1

- **Very thorough testing of MCNP6.1 & MCNP6.1.1 on many computer platforms:**
 - Brown, “MCNP6 Optimization & Testing for Criticality Safety Calculations”, LA-UR-15-20422 (2015).
 - Brown, Kiedrowski, Bull, “Verification of MCNP6.1 & MCNP6.1.1 for Criticality Safety Applications”, LA-UR-14-22480 (2014).
 - Brown, Kiedrowski, Bull, “Verification of MCNP5-1.60 and MCNP6.1 for Criticality Safety Applications”, LA-UR-13-22196 (2013).

Conclusion: **MCNP6.1 & MCNP6.1.1 are solid & reliable for criticality safety calculations**

- **MCNP6.1 is 20-30% slower than MCNP5,
MCNP6.1.1 is 10% faster than MCNP5**

User Support & Training

- **User support**
 - MCNP Forum - User-group, beginners & experts, >1000 members
 - MCNP Website, MCNP Reference Collection
 - Summer students
 - Direct hands-on support for LANL NCS Division
 - Email consulting to many crit-safety analysts
- **Classes**
 - **Theory & Practice of Criticality Calculations with MCNP**
 - FY13: 3 classes (including special class for LANL NCS group certification)
 - FY14: 2 classes (with some LANL NCS staff)
 - FY15: 2 classes (with some LANL NCS & DOE/NNSA staff), possibly other sites
- **Conferences & Journals**
 - M&C 2015, ICNC 2015, SNA+MC 2013
 - ANS Washington, Reno, Anaheim, San Antonio
 - OECD/NEA/WPNCS Expert Groups
 - Advanced Monte Carlo Techniques, Sensitivity/Uncertainty

Performance Improvements

- **MCNP6.1**

- Last few years – other developers focused on features, merger, testing, release
- Slower, by 30-500 %

- **Path forward – MCNP 2020**

- Concerted effort to modernize the codebase, upgrade foundations
- Goals: **faster, sustainable, flexible**
- Necessary for MCNP to survive into the 2020's & new computers
- Proposed joint support by DOE-ASC & DOE-NCSP
 - Experienced Lead (Brown)
 - 2-3 core developers

MCNP 2020

- **Improve performance**

- **Goal: 2X speedup within 2 years**

- **Upgrade core MCNP6 software**

- Restructure, clean up coding, Fortran 2003 & C/C++ standards
- Reorganize data structures
- Evolution, not revolution
- Reduce future costs for new development & maintenance
- **Goal: sustainable code**

- **Prepare for future**

- New computers – massive parallel, but less memory per core
- Improve MPI & thread parallelism
- **Goal: flexible, adaptable code**

- Initial 3-month effort, focus on speedup & optimization
 - Focus on neutron criticality problems common to ASC & NCSP applications
 - Speedup factors** from recent performance improvements, mcnp6.1.1:

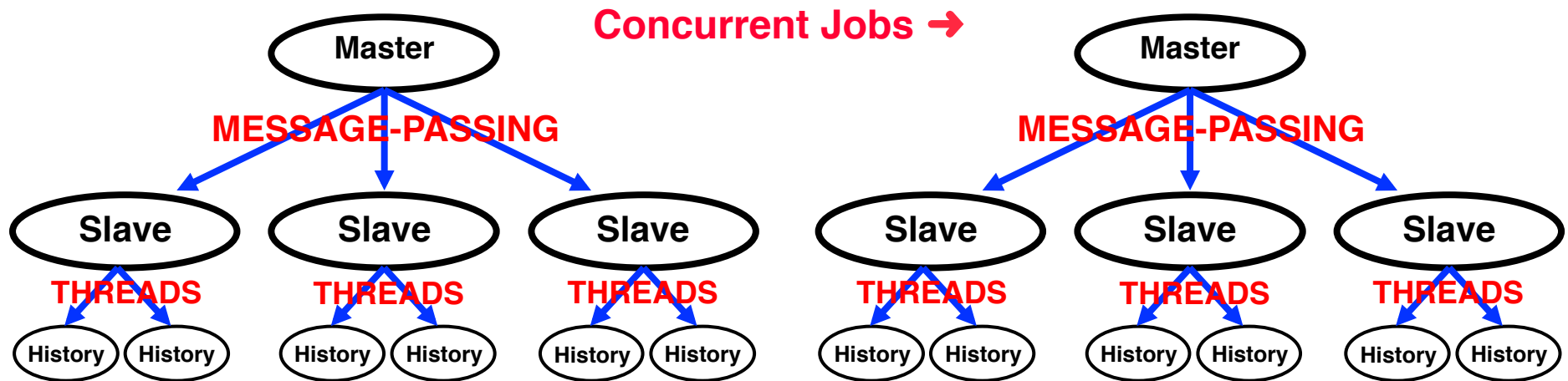
Performance Test Set			
Criticality		Other	
ks1	1.76	void1	3.03
ks2	2.13	void2	4.11
ks3	1.35	void2	4.11
ks4	1.36	void3	2.72
baw1	2.19	det1	1.67
baw2	1.59	med1	1.15
fvpf	2.04	pht1	1.22
g1	1.14		
g2	2.20		
pin	1.73		

VALIDATION_CRITICALITY Suite			
Measured wall-clock times, <u>including data I/O</u> :			
mcnp5	release	34.7	min
mcnp6.1	release	43.9	min
mcnp6.1.1	NEW	27.9	min

- 1.57 X speedup over mcnp6.1
- 1.24 X speedup over mcnp5

Performance Benchmark Suite	
Speedups vs MCNP6.1 Release	
Neutron Problems	Speedup
BAWXI2	4.37
GODIVA	1.05
Mode n in air w 750,000 tally bins	1.18
Well log problem	1.91
100M lattice cells in void	5.17
Other	
mode p e in air	1.01
mode n p e in air	1.05
mode p in air	1.20
Pulse height tally	1.20
Radiography	1.07

MCNP – Hierarchical Parallelism – Since 2000



Parallel Processes

- **Total processes = (# jobs) x (# MPI processes) x (# threads)**
- **Tradeoffs:**
 - More MPI processes - lots more memory & messages
 - More threads - contention from lock/unlock shared memory
 - More jobs - system complexity, combining results

MCNP 2020 - Performance Improvements (4)



Run Times for VALIDATION_CRITICALITY Suite on Various Computers

Computer	CPU Speed (GHz)	Mem. Speed (GHz)	Processors, Cores	MCNP Threads used	MCNP Version	Total Time (minutes)
MacBook 2010	2.7	1.1	1 - i7, 2 x 2 HT	4	mcnp6.1.1	88
MacBook 2013	3.0	1.6	1 - i7, 2 x 2 HT	4	mcnp5-1.60	40
				4	mcnp6.1	62
				4	mcnp6.1.1	42
Mac Pro 2010	3.0	0.67	2 - Xeon, 4	8	mcnp5-1.60	30
				8	mcnp6.1	44
				8	mcnp6.1.1	28
Windows 2012	2.7	1.3	2 - Xeon, 6	10	mcnp6.1.1	19
Mac Pro 2012	2.4	1.07	2 - Xeon, 4 x 2 HT	16	mcnp5-1.60	25
				16	mcnp6.1	32
				16	mcnp6.1.1	22
Mac Pro 2014	2.7	1.6	1 - Xeon, 12 x 2 HT	12	mcnp5-1.60	14
				12	mcnp6.1	20
				12	mcnp6.1.1	14
				14	mcnp6.1.1	12



Work in Progress

Sensitivity/Uncertainty Methods

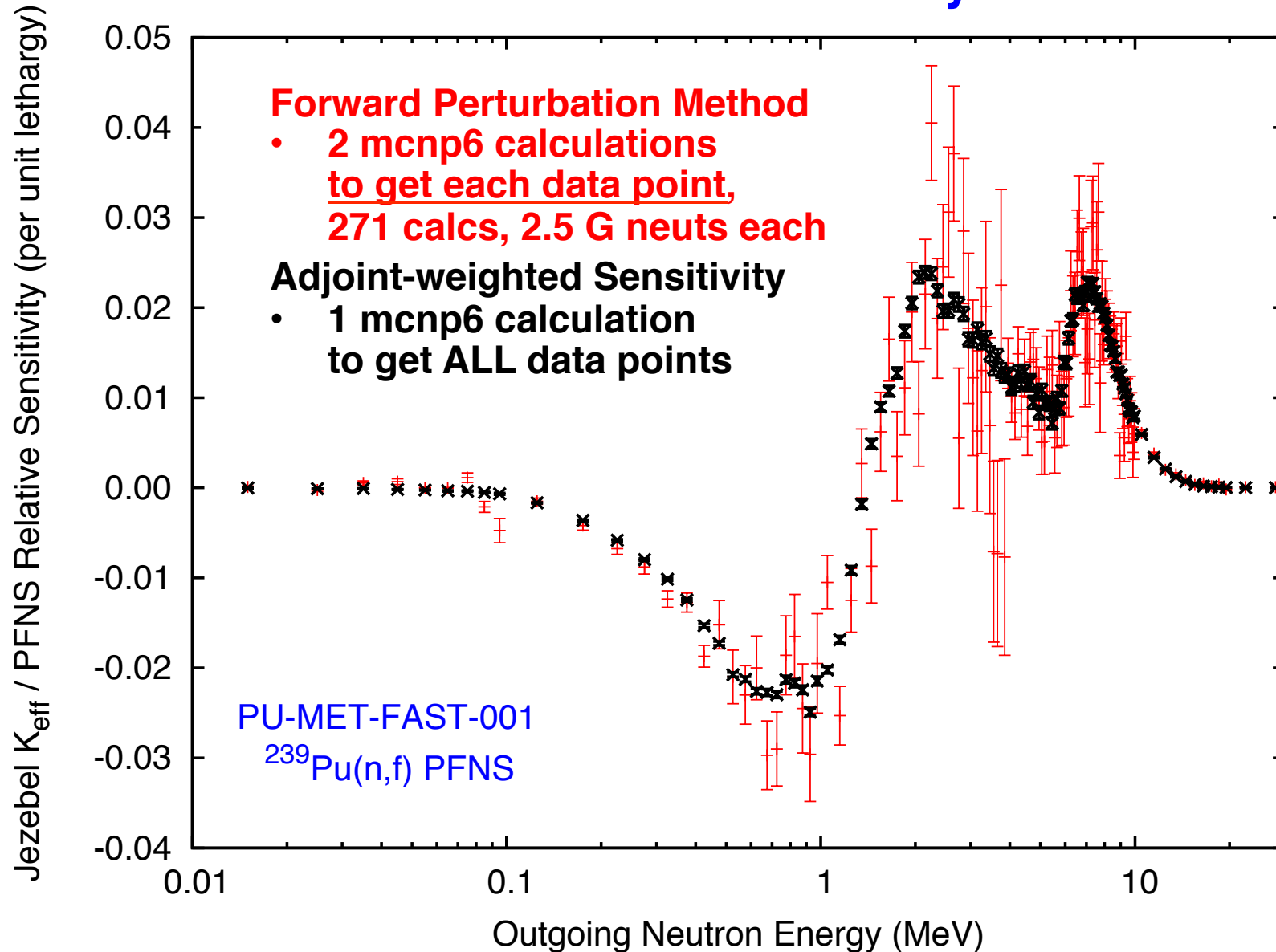
Whisper – Validation & USLs

MCNP 2020 – Near-Term Targets

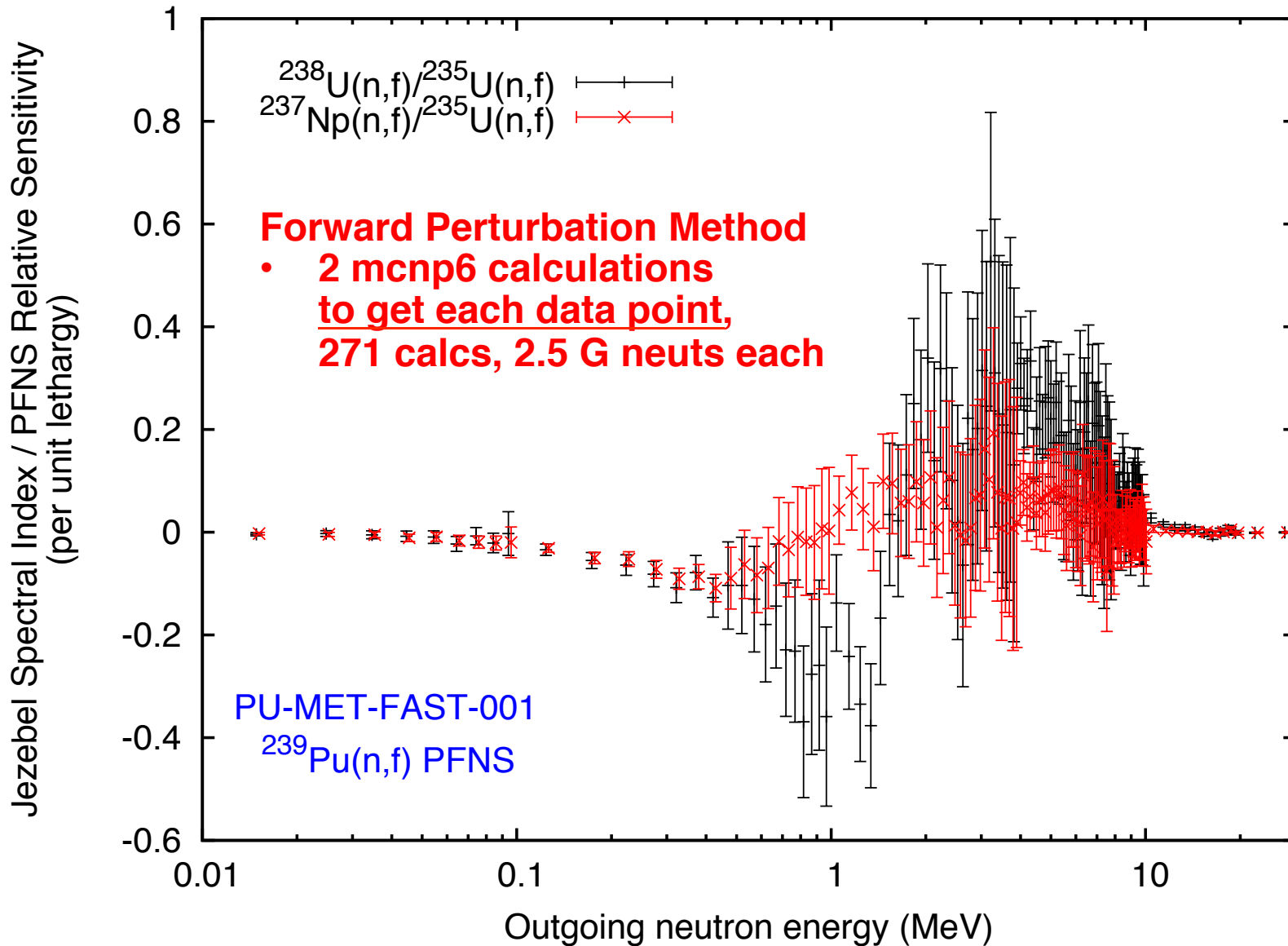
Other R&D Work, with Universities

- Adjoint-weighted sensitivity coefficient tallies can be used to predict the impact on K_{eff} due to newly evaluated nuclear data
- With a recent evaluation of the $^{239}\text{Pu}(n,f)$ prompt fission neutron spectrum (PFNS), we can quickly validate this new data against criticality safety benchmark simulations
 - Adjoint-weighted sensitivity calculation
 - 1 MCNP6 calculation for adjoint-weighted results at all energy points in the sensitivity energy range
- Some alternative approaches give poor results with more computational cost
 - Forward perturbation method
 - Direct, simple, brute force – subtract base case & perturbed case for each energy point in the sensitivity energy range
 - Requires 100s or 1000s of separate MCNP calculations, with small statistics
 - Examples on next slides required 2 mcnp6 calculation for each data point, 271 calculations with 2.5 G neutrons each

Jezebel – K_{eff} Sensitivity to PFNS



Jezebel – Spectral Indices Sensitivity to PFNS



- Other integral & semi-integral data have been measured on critical experiments such as,
 - A variety of spectral indices (ratios of reaction rates)
 - Overall and energy-dependent leakage
- When new nuclear data libraries are released (along with covariances), we should always
 - compare the calculated K_{eff} to experiment
 - compare the calculated spectral indices, leakages, etc., against experiment to provide feedback to the nuclear data community
- MCNP6 needs to be able to compute **adjoint-weighted sensitivity tallies for quantities other than k_{eff}** , such as spectral indices, to compare against this other experimentally measured criticality experiment data because the alternative approaches perform very poorly in comparison
- This capability is under development. May be especially important for supporting the CIELO nuclear data evaluations.

- In early 2014, the XCP-3 & NCS groups at LANL undertook a major upgrade to the criticality safety computational capabilities
 - Previous: mcnp5-1.25, endf 4, 5, 6 (very old & unsupported)
 - Upgrade: mcnp6.1 + endf/b-vii.1, HPC cluster
 - Participants:
 - Kiedrowski, Conlin, Favorite, Kahler, Kersting, Parsons, Walker, Brown, etc.
 - References
 - LA-UR-14-26558, *Whisper: Sensitivity/Uncertainty-Based Computational Methods and Software for Determining Baseline Upper Subcritical Limits*
 - LA-UR-14-26436, *User Manual for Whisper (v1.0.0), Software for Sensitivity- and Uncertainty-Based Nuclear Criticality Safety Validation*
 - LA-UR-14-23202, *Methodology for Sensitivity and Uncertainty-Based Criticality Safety Validation*
 - LA-UR-14-23352, *Validation of MCNP6.1 for Criticality Safety of Pu-Metal, -Solution, and -Oxide Systems*

- **Whisper ICSBEP Benchmark Suite**
 - 1086 ICSBEP benchmark problems from Mosteller, Kahler, others
 - Sensitivity profiles from adjoint-weighting for all isotopes/reactions/benchmarks
- **Whisper methodology** – LA-UR-14-26558, LA-UR-14-26436, LA-UR-14-23352
 - **Verification of computer code system**
 - Installation tests, VERIFICATION_KEFF tests, config control, static linked, etc.
 - **Validation benchmarks**
 - Estimate missing uncertainties
 - Reject inconsistent benchmarks via iterated diagonal chi-squared method (~12%)
 - Correlation data from DICE; covariance data from ORNL (10% diag for missing)
 - Automated benchmark selection for AOA problem using sensitivity data to determine C_k values; C_k values used for weighting
 - **Calculational Margin**
 - Determine bias from non-parametric method based on Extreme Value Theory, using weighting determined from C_k values
 - Determine bias uncertainty numerically from distribution of worst-case k_{eff} bias
 - **Margin of Subcriticality**
 - Margin of 0.0050 for unknown code errors (expert judgment)
 - Margin for nuclear data uncertainty from GLLS method
 - Additional margin – analyst judgment for AOA & problem, conservatism, etc.
 - **USL = 1.0 – Calculational Margin – Margin of Subcriticality**

- **Current activities**

- **NCS Division SQM for Whisper (XCP-3 assisting in review)**
 - *NCS-SQM Whisper Code Inspection (Sartor, in preparation)*
 - *NCS-SQM Whisper Verification & Validation (Sartor, in preparation)*
 - *NCS-SQM MCNP6 KCODE Verification & Validation (Sartor, in preparation)*
- **Whisper software**
 - Potential use at other DOE sites
 - Well-documented and tested alternative to tsunami/tsurfer/etc
 - **To be included with standard MCNP6 distribution through RSICC**
- **Whisper benchmark suite**
 - MCNP input for 1086 ICSBEP benchmarks
 - Valuable resource for all MCNP criticality-safety users & sites
 - **To be included with standard MCNP6 distribution through RSICC**
- **Improved covariance data – produce with NJOY & new ACE formats**
 - **Minor mods to Whisper, when Nuclear Data Team produces improved data**
- **Whisper training**
 - **Proposed to DOE-NCSP for LANL & other DOE crit-safety groups**
 - Local training at LANL (not DOE-NCSP funded)

NCSP-Related

- **Parallel threading** – measure 2015 performance of atomic-operations vs critical-sections
- **List tallies** – alternative tally scheme, to save memory & reduce lock/unlock overhead for threading
- **Light-weight cycle rendezvous for MPI calculations** – reduce unnecessary MPI messaging
- **Compliance with Fortran-2003 standard** – eliminate all coding using older or nonstandard features
- **Fission neutron multiplicity** – restructure & combine, ensure correct threading

Depending on non-NCSP funding

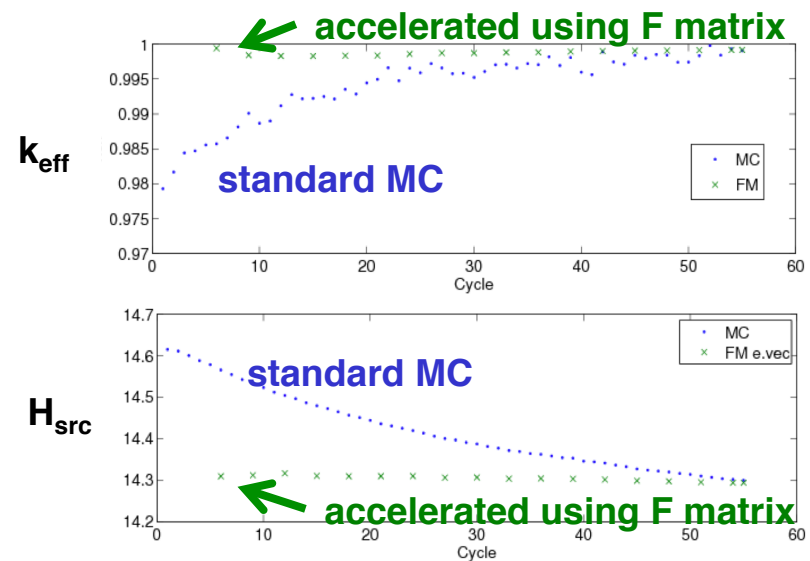
- **MPI improvements** – nonblocking messages, asynchronous transfers, in-place transfers, improved interface using Fortran-2003 polymorphism
- **Examine memory allocation** – rearrange for better cache utilization
- **Tally servers** – remote node storage for tallies with very large memory requirements
- **New standardized dump file** – direct access, access by dataset name, etc.
- **Upgrade for PTRAC & SSR** – permit use in parallel calculations, not just serial
- **HDF5 &/or MPI-IO** – improve read/write speed & portability of file output

Physics & Temperature Dependence

- Full temperature dependence of $S(a,b)$ thermal scattering (RPI)
- Unresolved resonances (MIT)
- Implement modified free-gas scatter, to model resonance upscattering for epithermal neutrons (Michigan)
- Investigate coupling MCNP into multiphysics calculations (Michigan)
- V&V for using explicit fission neutron multiplicity distributions in criticality calculations (New Mexico)
- Doppler coefficients (New Mexico)

Fission Matrix

- Forward & adjoint methods, sparse matrix schemes (Michigan)
- Automatically determine source convergence, without user input
- Apply to subcritical multiplication problems
- Accelerate source convergence



Summary

- MCNP6.1, MCNP6.1.1, & ENDF/B-VII.1 released
- Next release – TBD, probably FY 2016
- Impact on Criticality Calculations → **none**
 - All basic KCODE criticality features same as for MCNP5
 - Matches results with MCNP5 for criticality suites
- MCNP6 speed improved by 1.2 – 4 X for crit-safety.
- More MCNP 2020 improvements in progress
- Sensitivity/uncertainty methods based on adjoint-weighted tallies are being used routinely in many areas
 - Outstanding success due to long-range vision & support from NCSP
- Whisper methodology for validation & USLs is important to LANL NCS, and to other DOE sites
- Criticality-safety community needs to transition to MCNP6 over the next few years

Questions ?